

#### PHYSICAL MODELLING AND ADVANCED SIMULATIONS OF LIQUID VAPOR TWO-PHASE FLOWS WITH NEPTUNE\_CFD CODE

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#### OUTLINE

1. CFD MODELLING OF <u>DISPERSED TWO-PHASE FLOW</u> FOR NUCLEAR POWER PLANT (6-

EQUATION MODEL, EULER-EULER, NEPTUNE\_CFD (EDF,CEA,IRSN,FRAMATOME))

RECENT ADVANCES IN NUCLEATE BOILING MODELLING

**2.** MULTIFIELD CFD CALCULATIONS OF INDUSTRIAL GEOMETRIES : <u>LARGE INTERFACES</u>

FLOW PATTERN MAP : METERO, MAXI2 EXPERIMENTS

TURBULENCE

PHASE CHANGE

CAPILLARY AND WETTABILITY EFFECTS

- **3.** TWO-PHASE FLOW IN CRACKS OR LEAKY RODS
- 4. CFD MODELLING OF <u>DISPERSED TWO-PHASE FLOW</u> FOR NUCLEAR POWER PLANT (6-EQUATION MODEL, EULER-EULER, NEPTUNE\_CFD (EDF,CEA,IRSN,FRAMATOME)) MODELLING OF SPRAYS

Void fraction

1. CFD modelling of dispersed two-phase flow for nuclear power plant

## RECENT ADVANCES IN NUCLEATE BOILING MODELLING AND APPLICATION TO DNB FOR UNIFORM AND NONUNIFORM HEAT FLUX

IMFT, CEA, IRSN, FRAMATOME European projects : HZDR, GRS, PSI, KTH, UCL, JSI, VTT, UJV, KFKI, ...





## **DNB : INDUSTRIAL CONTEXT**

In nucleate boiling, heat flux increases and reaches a maximum value with increasing wall temperature.

ightarrow severe damage or meltdown of the surface.

A vapour film isolates the fuel from the water: the fuel heats up sharply and suddenly







#### THE NEPTUNE\_CFD SOLVER AND PHYSICAL MODELLING : BALANCE EQUATIONS : CFD 3D

Two mass balance equations:

$$\frac{\partial \alpha_k \rho_k}{\partial t} + \nabla (\alpha_k \rho_k \underline{V}_k) = \Gamma_k$$

The interfacial transfer terms of mass, momentum and heat.

Two momentum balance equations:

Reynolds stress tensor

$$\frac{\partial \alpha_k \rho_k \underline{V}_k}{\partial t} + \nabla . (\alpha_k \rho_k \underline{V}_k \underline{V}_k) = -\alpha_k \underline{\nabla} p + \underline{M}_k + \alpha_k \rho_k \underline{g} + \nabla . [\alpha_k (\underline{\Sigma}_k + \underline{R}_k)],$$

Two total enthalpy balance equations:

$$\frac{\partial}{\partial t} \left[ \alpha_k \rho_k \left( h_k + \frac{V_k^2}{2} \right) \right] + \nabla \left[ \alpha_k \rho_k \left( h_k + \frac{V_k^2}{2} \right) V_k \right] = \alpha_k \frac{\partial p}{\partial t} + \alpha_k \rho_k \underline{g} V_k$$
Wall transfer model for nucleate boiling

$$+\Gamma_{k}\left(h_{ki}+\frac{V_{k}^{2}}{2}\right)+\Pi_{k}A_{i}+q_{wk}^{\prime\prime\prime}-\nabla\left[\alpha_{k}\left(q_{k}+q_{k}^{\prime\prime}\right)\right]$$

turbulent heat flux

 $\Gamma_{l} = -\Gamma_{v} = \frac{\Pi_{l} + \Pi_{v}}{h_{vi} - h_{li}} A_{i} \quad \text{energy jump condition} \rightarrow \text{mass transfer term}$ 

#### FORCES EXERTED ON BUBBLES

$$\begin{split} \underline{M}_{g}^{D} &= -\underline{M}_{l}^{D} = -\frac{1}{8} A_{l} \rho_{l} C_{D} | \underline{V}_{g} - \underline{V}_{l} | (\underline{V}_{g} - \underline{V}_{l}) = -\alpha_{g} \rho_{l} F_{D} (\underline{V}_{g} - \underline{V}_{l}) \text{ Brag force} \\ \text{drag coefficient for bubbles has been empirically modelled by Ishir (1990):} \\ C_{D} &= \frac{2}{3} d \sqrt{\frac{g|\rho_{s} - \rho_{l}|}{\sigma}} \left( \frac{1 + 17.67(f(\alpha))^{g(2)}}{18.67f(\alpha)} \right) \text{ with } f(\alpha) = (1 - \alpha)^{15} \text{ for distorted bubbles} \\ C_{D} &= \frac{8}{3} (1 - \alpha)^{2} \text{ for chum-turbulent regime} \\ \\ \underline{M}_{g}^{AM} &= -\underline{M}_{l}^{AM} = -C_{A}^{Ig} \frac{1 + 2\alpha_{g}}{1 - \alpha_{g}} \alpha_{g} \rho_{l} \left[ \left( \frac{\partial \underline{V}_{g}}{\partial} + \underline{V}_{g} \cdot \underline{\nabla}\underline{V}_{g} \right) - \left( \frac{\partial \underline{V}_{l}}{\partial} + \underline{V}_{l} \cdot \underline{\nabla}\underline{V}_{l} \right) \right] \text{ Added mass force} \\ \text{added mass coefficient which is equal to ½ for a spherical bubble and the factor (1 + 2\alpha)/(1 - \alpha) takes into account the effect of the bubbles concentration} \\ \\ \underline{M}_{g}^{L} &= -\underline{M}_{l}^{L} = -C_{L}\alpha_{v}\rho_{l}(\underline{V}_{g} - \underline{V}_{l}) \land (\underline{\nabla} \wedge \underline{V}_{l}) \\ \text{empirically modelled by Tomiyama = f(EOH)} \qquad E_{O_{H}} = \frac{g(\rho_{g} - \rho_{l})d_{H}^{2}}{\sigma - d_{H}} = D_{g} \cdot \frac{1}{\sqrt{1 + 0.163E0^{0.15}}} \\ \text{where } d_{H} \text{ is the maximum horizontal dimension of the deformed bubble, which is calculated using an empirical correlation given by Wellek} \\ \\ \underline{M}_{g}^{TD} &= -\underline{M}_{l}^{TD} = -F^{TD} \rho_{l} \ k_{l} \nabla \alpha_{g} , \end{aligned}$$



Lavieville (2016)

#### WALL TRANSFER MODEL FOR NUCLEATE BOILING

In a first simplified approach, and following the analysis of Kurul (Kurul, 1990), the heat flux at the wall is split into three terms:

a single phase flow convective heat flux  $q_c$  at the fraction of the wall area unaffected by the presence of bubbles,  $q_c = A_c h_{log} (T_w - T_l)$ 

a quenching heat flux  $q_q$  where bubbles departure brings cold water in contact with the wall periodically,  $q_q = A_b t_q f \frac{2\lambda_l (T_w - T_l)}{\sqrt{\pi a_l t_q}}$ 

a vaporisation heat flux  $q_e$  needed to generate the vapour phase.

$$q_e = f \frac{\pi a_d}{6} \rho_v \ell n$$



*A<sub>b</sub>* : wall fraction occupied by bubble nucleation bubble detachment frequency bubble detachment diameter active nucleation sites density

\_1 3

#### → Empirical correlations

#### MODELLING OF THE LIQUID TURBULENCE ? →LIQUID TEMPERATURE AND VOID FRACTION





#### TOWARDS DNB MECHANISMS : WALL FUNCTION FOR BOILING FLOWS

At subcooled flow boiling, the liquid velocity profile in the boundary layer is significantly disturbed by the bubble formation and detachment mechanisms on the heated wall. In the literature an over-prediction of liquid and gas velocity distributions in the boiling boundary region has been reported.

Roy et al (2002) : ASU experiment

$$u^{+} = 1.8(\pm 0.25) \ln(y^{+}) + 5.9(\pm 1.0)$$
$$T^{+} = 1.95(\pm 0.15) \ln(y^{+}) + 6.2(\pm 1.2)$$





#### **DNB MODELING**

<u>S. Mimouni, "Computational multi-</u> <u>fluid dynamics predictions of critical</u> <u>heat flux in boiling flow</u>" *Nuclear Engineering and Design*, *Volume 299, 1 April 2016, Pages* 28-36

- Sensitivity to the mesh refinement
- Control of the oversaturation : If the liquid temperature in the nearest cell at the wall tends to the saturation temperature :

total heat 
$$flux = q_e + q_C + q_Q \rightarrow q_e$$

• Generalization of the Kurul-Podowski model :  $(q_v = h_{vap}(T_{wall} - T_v))$  heat flux,  $q_v$ , is the diffusive heat flux used to preheat the vapor phase :-----

$$\begin{aligned} q_{wall} &= g_{\alpha,A} \left( q_c + q_q + q_e \right) + \left( 1 - g_{\alpha,A} \right) q_v \quad \left[ W / m^2 \right] \\ \text{Nucleate boiling regime} \quad \text{Vapor film} \\ g_{\alpha,A} &= f_\alpha \quad \rightarrow \quad g_{\alpha,A} = f_\alpha \cdot f_A \end{aligned}$$

if  $T_l(y^+)$  reaches the saturation temperature :  $\alpha_{crit}=0.5$  and  $A_{crit}=0.5$ 

otherwise  $\alpha_{crit}=0.2$  and  $A_{crit}=0.2$ 



## **CALCULATIONS OF DNB TESTS IN A TUBE**

Russian Academy of Sciences produced a series of standard tables of CHF as function of the bulk mean water condition and for various pressures and mass velocities for fixed tube diameter of 8 mm (Groeneveld, 1996). Heated length = 1m.



Calculation is started with wall heat flux equal to 70%CHF. Wall heat flux is after increased of 5% progressively. After this stage, wall heat flux reaches a plateau in order to stabilize the boiling flow. This procedure is repeated. In the calculations, CHF is detected when the wall temperature increases sharply.

Because of the sudden rise in temperature , results are weakly sensitive to the wall temperature chosen for CHF detection. (sudden drop of the vaporization heat flux).



#### **1500 VALIDATION CASES**



#### **1500 VALIDATION CASES**



## Conclusion for the Critical heat flux

The objective of this work is to propose a new model in a computational multi-fluid dynamics tool leading to wall temperature excursion and onset of boiling crisis.

Critical heat flux is calculated against 1500 tests. The model tested covers a large physics scope in terms of mass flux, pressure, quality and channel diameter. Water and R12 refrigerant fluid are considered.

Furthermore, it was found that the sensitivity to the grid refinement was acceptable.

Neptune\_CFD code with the DNB model is currently assessed in the nuclear industry for design optimization of rod bundles.

**CFD results < empirical correlations based on experimental data** 

# How can we improve the model ? →Luc Favre, PhD 2023



## NUCLEATION FREQUENCY





Tsat

 $T_w$ 

$$\rightarrow f = \frac{1}{t_g + t_{wait}}$$

Boundary layer reconstruction New nucleation *t* = *tg*,*d* + *twait* 

# BUBBLE LIFT-OFF 1/2 : SLIDING LENGTH

#### Transient conduction induced by bubble sliding [Kossolapov, 2020]



Exemple de pied de page (A modifier dans l'onglet "Insertion"/"En-tête/Pied"

 $D_b = 2,42 \times 10^{-5} P^{0,709} \frac{a}{\sqrt{b\varphi}}$ 

## BUBBLE LIFT-OFF 2/2



#### **Force balance sketch**



# SITE DENSITY

#### $\Box RPI : N_{sit} = (210(T_w - T_{sat}))^{1,8}$

New phenomena are taken into account





- 1. Nucleation Site Density Nsit : empirical relation f(Tw, Tsat, P, contact angle)
- 2. Static growing bubble <u>overlapping</u> probability : Pcoal,st
- 3. <u>Static coalescence</u> site density : Ncoal,st = Pcoal,st (Nb,Rd) Nsit,a
- 4. →Static & <u>Sliding Coalescing</u> Site Density Ncoal,st & Ncoal,sl (Rsl, ...)

X Active site × : Deactivated site



Probability to find dry area at a particular spot on the boiling surface

10-1

#### Nucleation site distribution

10-2



 $q_{c,L} = Ac, I hc, I (Tw - TL) / \phi c, V = Ac, V hc, V (Tw - Tsat)$ 

• Static Coalescence Boiling Heat Flux :

$$\boldsymbol{q_{e,coal\,st}^{\prime\prime}} = \frac{N_{coal,st}}{2} f \rho_V h_{LV} \frac{4}{3} \pi R_d^3$$

• Sliding Coalescence Boiling Heat Flux  $q_{e,coal\,sl}^{\prime\prime} = \frac{N_{coal,sl}}{2} f \rho_V h_{LV} \frac{4}{3} \pi (R_{sl}^3 + R_d^3)$ 

## QUENCHING HEAT FLUX





$$\begin{split} \phi_{q} &= A_{q} t_{q} f \frac{2 \lambda_{L} \left( \Delta T_{w} + \Delta T_{L} \right)}{\sqrt{\pi \eta_{L} t_{q}}} \\ \text{Quenching area: } A_{q} &= \underbrace{N_{coal,st} \pi R_{d}^{2}}_{\text{static coal.}} + \underbrace{\frac{N_{coal,sl}}{2}}_{\text{bubble sliding area}} \end{split}$$

#### **NEW NUCLEATE BOILING : DEBORA EXP.**

Test case	Deb5	Deb6
Inlet mass flow rate (kg.m <sup>-2</sup> s <sup>-1</sup> )	1996	1984.9
Inlet temperature (°C)	68.5	70.5
Wall heat flux (MW.m <sup>-2</sup> )	1.2	0.8
Pressure (Mpa)	2.615	2.615
Quality	0.058	0.0848

**Boundary Conditions:** 



- 40 radial cells (~0.24 mm) <sup>•</sup> Axysymm
- 400 axial cells (~1 cm)
- Wall distance : y+ ~100
- n) Axysymmetric simulation
  - Outlet: uniform pressure
  - Inlet: uniform velocity
  - Wall: modified logarithmic law for

bubbly flows [Mimouni et al., 2016] &

uniform heat flux

• Other: Symmetry



## **C800**



## 2. Multifield CFD calculations of industrial geometries

# FLOWS WITH LARGE INTERFACESMSME, IMFT, CEALIQUID - GAS





## **APPLICATION : COMPLEX FLOWS IN STEAM GENERATORS**



#### **MODELLING STRATEGY: MULTIFIELD APPROACH**



## LIQUID / VAPOR INTERFACE





#### LIQUID / VAPOR INTERFACE

Surface tension force, Brackbill et al. [1992]:

□ For deformable interfaces with a finite thickness

$$F_{CSF} = \alpha_k \, \sigma \kappa \nabla \alpha_k$$
 with  $\kappa = -\nabla \cdot \left(\frac{\nabla \alpha_k}{||\nabla \alpha_k||}\right)$  : theory

Drag force law: To couple the velocity of each field at the interface: subgrid model



[Brackbill, J.U. *et al.*, 1992, A continuum method for modeling surface tension, *J. Comput. Phys.*, Vol. 100, pp. 335-354]

## NEED OF THE INTERFACE LIQUID/VAPOR MODELS : CASE OF A RISING BUBBLE



The sharpening equation or the surface tension is sometimes forgot in industrial studies in order to save CPU time but the results could be not realistic.

# **METERO EXPERIMENT (CEA)**

- M. Bottin, J.P. Berlandis, E. Hervieu, M. Lance, M. Marchand, O.C. Öztürk, G. Serre, "Experimental investigation of a developing two-phase bubbly flow in horizontal pipe".
- This experiment has been developed in the frame of the NEPTUNE project, jointly developed by CEA, EDF, FRAMATOME and IRSN.



- The test section, 5.40 m long, has an inner diameter D = 0.1 m
- air injection tubes have been set to ensure uniform bubble injection in the inlet section.
- Inlet : water (0–5 m/s)+ air bubble (0–0.7 m/s).



## EXPERIMENTAL OBSERVATIONS

#### MSME, IMFT, CEA





### **METERO: FLOW PATTERN MAP FOR X/D = 40**



Transition from slug to stratified flow (TSS) transition from plug to slug flow (TPS) transition from buoyant bubble flow to stratified bubble flow (TBBSB) transition from stratified bubbles regime to plug (TSBP)



## PLUG FLOW REGIME: MEDIUM VALUE OF LIQUID MASS FLOWRATE

dispersed gas 1.000e+00 7.500e-01 5.000e-01 2.500e-01 0.000e+00

continuous gas

1.000e+00

7.500e-01 5.000e-01

2.500e-01 0.000e+00

JL = 2.12 m/s; JG = 0.1273 m/s

time = 0.00e+00

JL = 2.4 m/s; JG = 0.03m/s



Side view

top bubbles coalesce to form plugs→ intermittent regime

Top view





Ŷ,



Figure 9c: Bubble velocity at 40D (plug flow).

Figure 9d: Void fraction at 40D (plug flow).



## SENSITIVITY TO THE MESH REFINEMENT



#### TOWARD AN DIMENSIONLESS-NUMBERS-BASED MODEL

large bubble of a given length scale L.

Several mechanisms can destabilize the bubble :

• Effect of gravity on bubble deformation can be described by EOTVOS number:

$$Eo = \frac{\Delta \rho g L^2}{\sigma}$$

• WEBER number: relative importance of the fluid's inertia compared to its surface tension

We = 
$$\frac{\rho_l v^2 L}{\sigma}$$

• HINZE introduces a turbulent WEBER value, comparing eddies kinetic velocity to surface tension cohesion of a bubble :

We = 
$$\frac{\rho_l(u')^2 L}{\sigma}$$

 $\rightarrow$  Eddies which scale is the same as the bubbles can lead to distortion and breakup of the bubbles.


#### TOWARD AN DIMENSIONLESS-NUMBERS-BASED MODEL

surface tension energy of bubble  $\sigma 4\pi (rac{d}{2})^2$  is

the turbulent kinetic energy of a sphere which diameter is  $\frac{1}{2}\rho_l\frac{4}{3}(\frac{d}{2})^3$  the same as the bubble, able to destabilize it, is :

Bubble breakup is possible when these quantities are of the same order of magnitude  $\rightarrow$ 



criterion based on non-dimensional numbers can be more easily generalized to different fluids and different thermalhydraulics conditions

#### APPLICATION TO TUBES VIBRATION ANALYSIS IN STEAM GENERATOR : MAXI 2 EXPERIMENT (CEA)

- 3D two-phase R114 Freon (simulant fluid for water at high pressure)
- 40 rows of 5 tubes (adiabatic) inclined of 30° with the horizontal.
- Void fraction and gas velocity are measured along the line NS defined by x = 48.75mm and z = 276.36mm, and the line WE defined by y = 48.75mm and z = 276.36mm, i.e. between the 7<sup>th</sup> and 8<sup>th</sup> row tubes.





#### Validation of the two-phase numerical model MAXI2 Experiment Freon/Freon



**39** 39

#### MAXI : 3 FIELDS → QUITE ENCOURAGING





## INTERFACE LOCATING METHODS→ LES



[Vincent, S., Tavares, M., Fleau, S., Mimouni, S. *et al.*, 2016, *A priori* filtering and LES modeling of turbulent two-phase flows Application to phase separation, *Comput. Fluids*]



## **CONCLUSION FOR MULTIFIELD FLOWS**

- Large interfaces and dispersed bubbly + droplets in the same calculations.
- Reasonable accuracy for industrial cases with reasonable grid size (1mm) and reasonable CPU.
- Sensitivity to mesh refinement  $\rightarrow$  reasonable.
- dimensionless numbers  $\rightarrow$  encouraging results for the transition regimes
- Turbulence → transition regimes : How to calculate all subgrid terms ?
- How to combine LES for large interfaces (deterministic approach) + RANS for dispersed flow (stochastic approach) ?
- Transition regimes → large interface calculated accurately → AMR (adaptive Mesh refinement) : which degree of maturity?



### **PHASE CHANGE : « BRACKBILISATION »**



#### **VALIDATION : VAPOR FRONT**

1D tube with a heated wall . The liquid is overheated, and the wall temperature is equal to  $T_{sat} \rightarrow steam$  is at the saturation temperature





**edf** 

the liquid begins to boil at the interface, which induces a displacement of the steam/water interface

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#### SODIUM FAST REACTOR : GR19→KNS 37



 $\sigma$  sodium 1 bar=157,  $\sigma$  water 1 bar=73,  $\sigma$  water 150bar = 12  $\rightarrow$  large bubbles

 $\rho l / \rho v = 2776$  (sodium) 1620 (water 1 bar) 6 (water 150 bar)



eDF

### 3. Multifield CFD calculations of industrial geometries

## WALL CONDENSATION, WETTING, CAPILLARITY AND DYNAMIC OF THE TRIPLE LINE

MSME, EDF (ERMES-MMC), ESPCI ANR MACENA 2 : I2M Bordeaux, INP Grenoble, SIAME Anglet



## **CAPILLARITY EFFECTS**



Surface tension force: 
$$F_{CSF} = \alpha_k \sigma \kappa \nabla \alpha_k$$
 with  $\kappa = -\nabla \cdot \left( \frac{\nabla \alpha_k}{||\nabla \alpha_k||} \right)$ 

In order to compute more precisely the interface curvature, we diffuse the interface:

$$\underbrace{\Delta \alpha_k}{\Delta \tau} - \nabla D \nabla \alpha_k = 0 \qquad \longrightarrow \qquad \kappa = -\nabla \left( \frac{\nabla \alpha_{k,diff}}{\|\nabla \alpha_{k,diff}\|} \right) \qquad (48)$$

### WETTING EFFECTS



Diffusion equation generalized :

$$\frac{\Delta \alpha_k}{\Delta \tau} - \nabla . D \nabla \alpha_k + B^S (\alpha_k^{n+1} - \alpha_p) = 0$$



At the wall :  $B \rightarrow \infty$ Penalty term  $\alpha_{k;diff} \rightarrow \alpha_p$  at the wall.

#### CONCLUSION

- Models for disperse fields (bubbles and droplets) are available
- validation of the multifield approach : Verification cases, Validation cases, Integral validation cases
- Sensitivity to mesh refinement
- Phase changes : dispersed gas phase and continuous gas phase → SFR ... DNB, Steam Generator, ...
- Wettability and capillarity effects → Dynamics of capillary bridges in a crack

#### $\rightarrow$ new challenges ?





#### ALL regime flows for nuclear power plant





models presented previously work together.

#### **TWO-PHASE FLOWS IN CRACKS**

During the course of hypothetical accidents in a PWR  $\rightarrow$  large mass and energy releases into the containment (Steam Line Break, Loss of Coolant Accident, etc.),

Vapor  $\rightarrow$  P increases  $\rightarrow$  mass flow rate through the concrete ?





# ALL MODELS WORK TOGETHER: CRACKS IN THE CONCRETE

#### Head losses increase $\rightarrow$ flow rate decrease





#### **TWO-PHASE FLOWS IN CRACKS : VALIDATION CASE**

different regime flows  $\rightarrow$  all the models detailled work together.





#### I. STUDY CONTEXT: LEAKY RODS

 About ten rods crack each year in France.



Inside a rod

uranium pellet (Aubrun &

- Filling the free spaces of the leaky rods with water.
- Potential hydrogen risk (explosion) during the evacuation of spent fuel to La Hague:
  Maginary José H2O → H2 + ½ O2



#### II. NUMERICAL STUDY: WETTABILITY

Experimental case of Mukherjee (2005) – representative of microchannel boiling (drying of leaky rods) Microchannel size 200  $\mu$ m x 260  $\mu$ m x 1200  $\mu$ m. Boundary conditions: Inlet (uin = 0.127 m/s and Tint = 102°C) – Outlet (P = 1 bar).

Contact angle  $\theta$  = 30°.







#### II. NUMERICAL STUDY: WETTABILITY





SedF

#### II. NUMERICAL STUDY: WETTABILITY



Quantitative results in accordance with experience.
Show the importance of considering

wettability.



### III. EXPERIMENTAL STUDY $\rightarrow$ SLUG regime

- Periodic nucleation of steam bubbles.
- Growth of bubbles by interfacial boiling as predicted by numerical simulations.







#### IV. Upscaling

- The calculations presented in Part II. are very expensive numerically.
- Need a very fine mesh.
- The  $\Delta t$  time steps are very small (in the order of 10-6 to 10-8 s) because of the CFL constraint :  $.CFL = \frac{u\Delta t}{\Delta x} < 1$

Simulation of one hour of drying of a leaky rod:

3.2 billion cells.

11,415 years on 64,000 processors.

#### The objective is to transform a 3D simulation into an equivalent 1D simulation. Simulation 3D







### 4. CFD modelling of dispersed two-phase flow for nuclear power plant

## MODELLING OF SPRAYS IN A MULTI-COMPARTMENT GEOMETRY WITH A CMFD CODE

IMFT, IRSN, ESPCI European projects : PSI, UCL, JSI, Becker T., KFKI, U. Pisa, CEA, ...



#### BACKGROUND

hypothetical accidents in a PWR : Steam Line Break  $\rightarrow$  lead to large mass and energy releases into the containment

- $\rightarrow$  spray systems are used in the containment:
- in order to limit overpressure
- to enhance the gas mixing in case of the presence of hydrogen
- to drive down the fission products





Computation of Sh and Nu numbers : relations of Frössling / Ranz-Marshall Tabulated laws :  $D(T_m),\,\rho_{sat}(T_2),\,\lambda_1(T_m)$ 



#### PANDA EXPERIMENT (PSI)





The spray nozzle is oriented vertically downward in vessel 1. It produces a conical solid spray pattern. The two vessels are connected with a 1 m diameter pipe (IP).

Ddroplet=0.582mm





#### Sectional method developped in NEPTUNE\_CFD code



← Monodispersed

 $\mathsf{Polydispersed} \rightarrow$ 



Cutting the size distribution into sections



Solving the equilibrium equations for each section:

- mass
- momentum
- enthalpy

#### Equation closure terms:

- turbulence (+ inverse coupling)
- drag (between sections and gaseous phase)
- collision terms (mass and momentum transfers)

1 size <-> 1 velocity

Development of the sectional approach into the NEPTUNE\_CFD code

#### **Droplet size**



polydispersed approach : 8,9



#### **COLLGATE : Modeling the droplet collision outcome**



#### **BC of the simulation**



Coarse mesh Standard mesh 700,000 cells 1,968,500 cells 3 m 3 m 7 m 7 m Wall Wall 3.5 m 3.5 m Wall Wall Wall Wall

Section	Diameter	Flowrate
1	55 µm	1.42 10 <sup>-5</sup> kg/s
2	166 µm	2.67 10 <sup>-2</sup> kg/s
3	277 µm	1.28 10 <sup>-1</sup> kg/s
4	388 µm	1.91 10 <sup>-1</sup> kg/s
5	500 µm	2.02 10 <sup>-1</sup> kg/s
6	611 µm	1.72 10 <sup>-1</sup> kg/s
7	722 µm	1.29 10 <sup>-1</sup> kg/s
8	833 µm	8.87 10 <sup>-2</sup> kg/s
9	<b>944 µm</b> Ste	ph <b>6.35</b> in10 <sup>-2</sup> kg/s

Experimental and numerical local size distributions obtained for two interacting sprays are compared for different positions along the symmetrical axis :

Inlet conditions: definition of 9 sections for each nozzle



#### **Two PWR interacting sprays: some results**

•The smallest droplets are drifted away in the air flow.

•The biggest droplets, having more inertia, are not altered in the spray interacting area.





before spray interaction and about 200

µm after spray interaction.

## VERCORS EXPERIMENT, A 1/3 MOCK-UP OF A 1300 MWE NUCLEAR REACTOR CONCRETE CONTAINMENT



- Built by EDF in order to investigate the behaviour of concrete containment building in scenarios where a large amount of vapor is released in the containment
- The containment is inially filled with air only at 1 bar. A steam mass flow rate is imposed at the bottom (6 tons/hour) in the internal containment. But, this vapor mass flowrate evolves during the transient in order to maintain a pressure equal to 5 bar in the containment
- The concrete width for the wall containments is about 40 cm



COUPLING BETWEEN 3 CODES : NEPTUNE CFD + NEPTUNE\_CFD FOR TWO DISCONNECTED FLUID COMPUTATIONAL DOMAIN AND SYRTHES IN THE WALLS






Before launching the experimental test, CFD calculations have been performed to assess the evolution of the wall temperature in the internal containment and the wall temperature of the external containment.



## Upper part of the inter-containment :gas temperature reaches 60 C after only 24h

#### → detrimental for temperature sensors;

Mid-level, the temperature is about 40 C at time = 24h which is acceptable.

- Moreover, the calculation demonstrates the feasibility of the tests regarding the vapor mass flow rate injected during the tests.
- The calculation estimates also the amount of liquid from condensation in the wall that needs to be evacuated.



#### CONTAINMENT BUILDING OF A 1300 MWE FRENCH PWR

- cylindrical shape with a maximum height of 59 m, a maximum diameter of 40 m and a volume of 70 437 m3.
- local mesh refinement have been performed in this study in the region of spray aspersion in order to reach a cell size of about 1 cm.
- One-dimension fluid-structure heat transfer model has been applied to several structures : enclosure covering the building (90 cm), handling bridge (70 cm) and internal walls (40 cm).









Hydrogen, vapor and water spray mass flowrate injected at the boundary condition.

Location of the recombiners of the 1300MWe unit

The mitigation of the hydrogen risk is ensured by a set of hydrogen recombiners installed in the enclosure.

$$H_2 + \frac{1}{2}O_2 \rightarrow H_2O$$

Droplet diameter is injected at 2 mm to avoid being in a situation favorable to the pressure drop, because small drops of the order 100 micrometers increase the surface of exchange in the thermal transfer. Vapor condenses onto droplets which modifies the droplet diameter as a function of time and space



## SENSITIVITY TO THE MESH REFINEMENT SENSITIVITY TO DROPLET ASPERSION<sup>Vithout recombiner (black)</sup>



Comparison between coarse and reference meshes on the simulation of enclosure pressure. <u>referent mesh with eas (blue), without eas (red)</u> and in coarse mesh with eas (blue thick) and without eas (red thick).

 $\rightarrow$  eas inhances homogenization of the hydrogen in the containment which increases H2 consumption by the recombiners



P [Pa]

### **CONCLUSION FOR SPRAY CASES**

- A spray modelling is available in the NEPTUNE\_CFD code.
- Validation cases : CARAIDAS, TOSQAN, PANDA, CALIST, MISTRA, COTHYD (see oral presention of Tian CHEN in CFD4NRS)
- Spray model extended to wall Condensation at the wall (+ H2 recombiner ): COPAIN, TOSQAN ISP47, PANDA 25, H2PAR, THAI HR49, ...





# Simulation of Flow-Accelerated Corrosion (FAC) using Computational Fluid Dynamics (CFD)

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Context,
Phenomenology,
& State of the Art

2. Numerical Methodology Step-by-Step

**3.** Conclusion & Perspectives





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#### 1 – Industrial Context

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Flow-Accelerated Corrosion (FAC) : chemical degradation accelerated by a flowing electronic





Fig: failure of the piping integrity due to exacerbated wall thinning.

- Local wall thinning prediction: major security and availability issue (+ environmental and economical impacts)
- Large application sectors impacted: energy production and transport, chemical industry, etc.
- Study environment: Pressurized Water Reactor 2<sup>nd</sup> circuit (non/low alloy steel, liquid-vapor flow, reducing+alkaline chemistry)

#### 1 – Scientific Lockups





#### 1 – Simulation: state of the art

## Increase use of CFD with various level of modelization:

- complex geometries, two-phase flows, local
- corrosion enhancement
  - function of the understanding of the underlying mechanisms

### Uncoupled « Classical » approach / elementary





**Hypothesis:** No assumption on the rate-limiting step. **Estimation:** From laws of the theory of electrochemistry.

Gas z = hA - Interfacial  $\mathbf{\mathbf{\mathbf{\psi}}}_{\dot{N}A,interf}$ Transfer D - Reaction B - Mass  $\dot{R}_A$ Homogeneous Transport  $N_{A,m}$ Liquid zC - Reaction Heterogeneous z = 0 $N_{A,r}$ 

**Fig:** In the coupled approach: mass transport of a passive scalar  $A : H^+, Fe^{2+}, ...$  with different fluxes and reactions.







#### 1 – Work on numerical methodology to apprehend FAC



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epredr

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#### 2 – Modelization of turbulent mass transport

#### Study case

- Regardless of the corrosion model: Wall Mass Transfer = key process → must be adequately captured.
- What are the turbulence RANS approaches/models valid to simulate this phenomenon ?





#### 2 approaches to deal with walls





#### Conclusion

Low-Re approach: wrong turbulent viscosity profile at wall vicinity = wrong concentration profile for  $Sc \gg 1$  (thin conc. BL).

#### 2 – Wall Mass Transfer of low and high Sc in rough pipes

**Method** : High-Re approach (roughness not resolved but modeled) → modified wall functions via "sand-grain roughness"



#### Conclusion

Clear improvements in the prediction for low and high Sc in all types of rough regimes: smooth/transitional/full-rough.

#### 2 – Mass Transfer: Extension to two phase (2-P) flow

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**GOAL** Validate Wall Mass Transfer in different two-phase flow regimes (methodology adaptation)



Fig: Boundary
conditions for mass
transfer

nsfer	and carrying phase	
Autor	Regime	Sc
Wang et al. (2002)	Liquid, slug	1620
Langsholt et al. (1997)	Stratified, slug	473
Pecherkin et al. (2007)	Liquid, bubbly	1500
Zheng et al. (2006)	Slug	1140
Mazhar et al. (2013)	Liquid, annular	1280





Fig: Final results - complete a prediction map for any 2-P flow regimes.

Tab: Experience list for validation of Wall Mass Transfer in 2-P flow.







Fig: Comparison: experiments / 0D-model / numerical simulations for different liquid  $(J_l)$  and gas  $(J_g)$  superficial velocities (mean values).





M. Bouchacourt, IWG-RRPC-88-1, 1988



Source of errors from the use of simple classical model ? → Roughness ? Lack of electrochemistry... ? Wrong rate limiting step ?

#### **2** – Comprehensive mechanistic model from $CO_2$ -corrosion to $2^{nd}$ coolant systems







#### 2 – Comprehensive mechanistic model: preliminary computations for model adjustment

Adapt chemistry (reactions, chemical species, rates constant...), then propose solution for new unknowns :



2 Solve equation set based on reactive mechanisms First qualitative results... ... still improvements Bouchacourt (1988) - Tube  $D_H = 8mm$ ; Ammonia (NH<sub>4</sub>)  $pH_{25^{\circ}C} = 9.0$ 8 Expe.  $U = 1m. s^{-1}$ Expe.  $U = 2m. s^{-1}$ Corrosion rate [mm/10<sup>5</sup>h] Expe.  $U = 3m. s^{-1}$ 6 Expe.  $U = 4m. s^{-1}$ CFD Smooth pipe 0 100 120 220 140 160 180 200 240 Temperature T [°C] Fig: Comparison of experimental data and

Fig: Comparison of experimental data and electrochemical

model of  $H_2O$  corrosion with oxide effect.





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#### **3 – Conclusion & Perspectives**

*Main objective:* develop a numerical methodology for FAC predictions valid in single and two-phase flow.

Proposed solution: mechanistic approach with coupled models of mass-

transport/electrochemistry integrated into CFD tools.

#### **Pure Mass Transfer - No reactions**

**1** – Comparisons of RANS turbulence modeling of Wall Mass Transfer: passive scalar at low/high *Sc*.

General conclusions

2 – Account for roughness effect which derives from surface

degradation (with High-Re/Wall Functions).

**3 –** Extend validation in two-phase flow conditions.

#### With reactions

**4** – Integration of a coupled mechanistic approach to predict corrosion in complex  $CO_2$  chemical system.

**5** – Preliminary results of *FAC* predictions extended to PWR secondary coolant conditions.

"Close term"

- perspectives
- Finalize: wall mass transfer in several roughness pattern, map of two-phase flow mass transfer.
- Improve mechanistic model for "secondary-water" environment (High-pH, large temperature range...).
- Exhaustive comparison: "mass-transfer controlled" and "all-coupled" modes of FAC predictions.

## CONCLUSION

Industrial needs: DNB, hydrogen risk, steam generator tube vibration, two-phase flows in cracks, sodium fast reactors, ...



Thank you for your

## attention

To my father (1939-2020)



## SHORT BIOGRAPHY



- engineering degree (M.Sc.) in aerospace in 1991 from ISAE. In 1996, Ph.D. in fluids mechanics from the Ecole Polytechnique ("fractal analysis of interfaces for the rayleigh-taylor instabilities"; Accreditation to supervise research (2018) Eiffel University.
- 2. over 100 publications in refereed journals and international congresses
- **3.** European projects : NURESIM, NURISP, NURESAFE, SARNET, SARNET2, SETH2 and HYMERES. He is currently implied in the european projects ESFR-SMART, SAMHYCO-NET, SAMOSAFER.
- 4. He has been the industrial/academic supervisor of 20 Ph.D.
- 5. He has been also the industrial supervisor of 35 M.Sc. Students.
- 6. Stephane Mimouni gave lectures in university Descartes (m.Sc.), INSTN (national institute of science and nuclear technology), Eiffel university, FJOH summer school, Mines Saint Etienne,...



## **EFFECT OF NON-UNIFORM AXIAL HEAT FLUX**

At PWR conditions, DNB will not occur at the tube exit with a cosinusoidal or skewed cosinusoidal flux distribution.

In this case, the DNB is

- in the high flux region
- governed by local conditions.





## EFFECT OF VOLUME FRACTION CRITICAL VALUE



